

(CLI-94-14) of September 2, 1994, the Rancho Seco decommissioning plan is approved and decommissioning of the Rancho Seco Nuclear Generating Station is authorized subject to the following conditions:

(A) The licensee shall update both the decommissioning plan and the final Safety Analysis Report (FSAR), currently entitled "Defueled Safety Analysis Report" or DSAR by the Rancho Seco licensee, in accordance with the methodology specified in 10 CFR 50.71(e), except that the frequency of updates shall be at least every 2 years. With respect to changes to the facility or procedures described in the updated FSAR/DSAR or changes to the Decommissioning Plan, and the conduct of tests and experiments not described in the FSAR/DSAR, the provisions of 10 CFR 50.59 shall apply.

(B) The licensee shall maintain a fire protection program to address the potential for fires which could result in a nuclear hazard, i.e., cause the release or loss of control of radioactive materials. The objectives of the fire protection program are to: (1) Reasonably prevent such fires from occurring; (2) rapidly detect, control, and extinguish those fires which do occur; and (3) ensure that the potential hazard due to fire to the public, environment, and plant personnel is small. The fire protection program shall be assessed by the licensee on a regular basis and revised as appropriate throughout the various stages of facility decommissioning. The licensee may make changes to the fire protection program without NRC approval if the changes do not reduce the effectiveness of fire protection measures needed to prevent a nuclear hazard at the Rancho Seco facility, taking into account the decommissioning plant conditions and activities.

Pursuant to 10 CFR 51.21, 51.30, and 51.35, the Commission has prepared an Environmental Assessment and Finding of No Significant Impact for the proposed action. Based on that assessment, the Commission has determined that the proposed action will not result in any significant impact on the quality of the human environment and that an environmental impact statement needed not be prepared. The Notice of Issuance of Environmental Assessment and Finding of No Significant Impact was published in the **Federal Register** on June 23, 1993 (58 FR 34065). Comments were received from ECO and are addressed in the supplemental safety evaluation issued concurrently with this Order.

For further details with respect to this action, see: (1) The application for authorization to decommission the facility, of May 20, 1991, as supplemented April 15, August 6, August 31, 1992, January 7, April 7, April 19, 1993, March 23, April 28, July 26, and October 26, 1994; (2) the related Commission safety evaluations dated June 16, 1993 and March 20, 1995; and

(3) the Environmental Assessment and Finding of No Significant Impact of June 16, 1993. These documents are available for public inspection at the Commission Public Document Room, the Gelman Building, 2120 L Street NW, Washington, DC 20555, and at the Martin Luther King Regional Library, 7340 24th Street Bypass, Sacramento, California 95825. Copies of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Project Support.

Dated at Rockville, MD, this 20th day of March 1995.

For the Nuclear Regulatory Commission,  
**William T. Russell,**  
*Director Office of Nuclear Reactor Regulation.*  
[FR Doc. 95-7572 Filed 3-27-95; 8:45 am]  
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#### [Docket No. 50-206]

#### **Southern California Edison Co., et al.; San Onofre Nuclear Generating Station, Unit No. 1**

Notice is hereby given that the United States Nuclear Regulatory Commission (the Commission) is reviewing the proposed corporate restructuring of San Diego Gas & Electric Company (SDG&E), one of the co-owners of San Onofre Nuclear Generating Station, Unit 1. By letter dated November 15, 1994, Richard A. Meserve of Covington & Burling, Counsel for SDG&E, informed the Commission that a corporate restructuring of SDG&E has been proposed that will result in the creation of a holding company under the temporary name SDO Parent Co., Inc. ("Parent Company") of which SDG&E would become a subsidiary. Under the restructuring, the holders of SDG&E common stock will become the holders of common stock of the Parent Company on a share-by-share basis. After the restructuring, SDG&E will continue to be a public utility providing the same utility services as it did immediately prior to the reorganization. SDG&E will continue to be a licensee of the San Onofre units, and no transfer of the operating licenses or interests in the units will result from the restructuring. Control of the operating licenses for the San Onofre units, now held by SDG&E and its co-owners, will remain with SDG&E and the same owners and will not be affected by the restructuring.

Dated at Rockville, MD, this 20th day of March 1995.

For the Nuclear Regulatory Commission.

**Seymour H. Weiss,**  
*Project Director, Non-Power Reactors and  
Decommissioning Project Directorate,  
Division of Project Support, Office of Nuclear  
Reactor Regulation.*

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#### [Docket No. 50-286]

#### **Power Authority of the State of New York; Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from the requirements of 10 CFR Part 50, Appendix R, "Fire Protection Program for Nuclear Power Facilities Operating Prior To January 1, 1979," Section III.J, to the Power Authority of the State of New York (the licensee) for the Indian Point Nuclear Generating Unit No. 3 (IP3), located at the licensee's site in Westchester County, New York. This exemption would be a modification of an exemption which has been previously issued on January 7, 1987.

#### **Environmental Assessment**

##### *Identification of Proposed Action*

Section III.J of 10 CFR Part 50, Appendix R, requires that emergency lighting units with at least an 8-hour battery power supply shall be provided in all areas needed for operation of safe shutdown equipment and in access and egress routes thereto.

During a programmatic review of Appendix R compliance strategy at Indian Point Nuclear Generating Unit No. 3 (IP3), the licensee identified that certain additional operator actions, which had not been included in the previous Appendix R compliance strategy, are needed. These additional operator actions are needed in the outside yard area at the condensate storage tank (RWST), and backup service water pump platform.

By letter dated January 7, 1987, the NRC previously approved an exemption from the emergency lighting requirements of Appendix R, Section III.J to allow the use of security lighting for access and egress to the Appendix R diesel generator which is also located in the outside yard. The license has, therefore, requested a modification to the exemption which was previously issued by the NRC on January 7, 1987, to extend the use of security lighting in the outside yard to include the CST,

RWST, and backup service water platform.

#### *Need for the Proposed Action*

The proposed exemption is needed to permit the licensee to restart and operate the plant without being in violation of the Commission's regulations and to obviate the need for extensive modifications. Physical modification of the plant to achieve literal compliance with 10 CFR Part 50, Appendix R, Section III.J, would require installation of emergency lighting in an exposed outdoor environment which would require special protective enclosures and extensive maintenance to maintain operability. Since security lighting which meets the requirements of 10 CFR 73.55(c)(5) is already installed in these areas and since this security lighting has already been acceptable to justify a similar exemption which was issued on January 7, 1987, application of the regulation in this particular circumstance would not be necessary to achieve the underlying purpose of the rule. The security lighting system achieves the underlying purpose of the rule in that it provides adequate illumination to perform the above-mentioned activities in the outside yard for a period of at least 8 hours, has its own independent emergency generator power source, and is not impacted by fires in other areas of the plant for which Appendix R fires need to be considered.

#### *Environmental Impacts of the Proposed Action*

The proposed action is to modify the exemption which was previously issued by the NRC on January 7, 1987, to extend the use of security lighting in the outside yard to include the CST, RWST, and backup service water platform. The Commission has completed its evaluation of the proposed action and concludes that the probability or consequences of accidents will not increase, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does involve features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes

that there are no significant nonradiological environmental impacts associated with the proposed action.

#### *Alternatives to the Proposed Action*

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

#### *Alternative Use of Resources*

This action does not involve the use of any resources not previously considered in the "Final Environmental Statement for the Indian Point Nuclear Generating Station Unit No. 3," dated February 1975.

#### *Agencies and Persons Consulted*

In accordance with its stated policy, the staff consulted with the State of New York regarding the environmental impact of the proposed action. The State official had no comments.

#### *Finding of No Significant Impact*

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated March 15, 1995, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street NW., Washington, DC, and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York.

Dated at Rockville, Maryland, this 23rd day of March 1995.

For the Nuclear Regulatory Commission.

**Daniel G. McDonald,**

*Acting Director Project Directorate I-1  
Division of Reactor Projects—I/II Office of  
Nuclear Reactor Regulation.*

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#### **Virginia Electric & Power Co.; North Anna Power Station, Unit No. 2; Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from Facility Operating License No. NPF-7, issued to Virginia Electric and Power Company (the licensee), for operation of the North Anna Power Station, Unit No. 2 (NA-2) located in Louisa County, Virginia.

#### **Environmental Assessment**

##### *Identification of the Proposed Action*

This Environmental Assessment has been prepared to address potential environmental issues related to the licensee's application of March 2, 1995. The proposed action would exempt the licensee from the requirements of 10 CFR Part 50, Appendix J, Paragraph III.D.1.(a), to the extent that a one-time interval extension for the Type A test (containment integrated leak rate test) by approximately 16 months from the March 1995 refueling outage to the October 1996 refueling outage would be granted. In addition, the proposed action would exempt the licensee from a portion of Section IV.A that requires a Type A test to be performed following a major modification or replacement of a component which is part of the primary reactor containment boundary.

Specifically, the post-modification exemption is requested from performing a Type A test due to the activities associated with the upcoming NA-2 steam generator replacement.

##### *The Need for the Proposed Action*

The proposed action is needed to permit the licensee to defer the Type A tests from the NA-2 March 1995 refueling steam generator replacement outage to the October 1996 refueling outage, thereby saving the cost of performing the test and eliminating the test period from the critical path time of the outage.

##### *Environmental Impacts of the Proposed Action*

The Commission has completed its evaluation of the proposed action and concludes that the proposed one-time exemptions would not increase the probability or consequences of accidents previously analyzed and the proposed one-time exemptions would not affect facility radiation levels or facility radiological effluents. The licensee will continue to be required to conduct the Type B and C local leak rate tests which historically have been shown to be the principal means of detecting